Standards Manager Web Standards List ANS-American Nuclear Society

Id	Number	Title	Year	Organization	Page
1	20.2	Nuclear Safety Design Criteria and Functional Performance Requirements for Liquid-Fuel Molten Salt Reactor Nuclear Power Plants	2023	ANS	
2	8.21	use of fixed neutron absorbers in nuclear facilities outside reactors	2023	ANS	
3	8.3	Criticality accident alarm system	2022	ANS	0
4	8.7	Nuclear Criticality Safety in the Storage of Fissile Materials	2022	ANS	0
5	RA-S-1.1	Standard for Level 1/Large Early Release Frequency Probabilistic Risk Assessment for Nuclear Power Plant Applications	2022	ANS	0
6	RA-S-1.4	Probabilistic Risk Assessment Standard for Advanced Non-Light Water Reactor Nuclear Power Plants	2021	ANS	0
7	3.14	Process for Infrastructure Aging Management and Life Extension of Nonreactor Nuclear Facilities	2021	ANS	0
8	55.1	Solid Radioactive Waste Processing System for Light-Water-Cooled Reactor Plants	2021	ANS	0
9	56.8	Containment System Leakage Testing Requirements	2020	ANS	0
10	18.1	Radioactive Source Term for Normal Operation of Light Water Reactors	2020	ANS	27
11	2.27	criteria for investigations of nuclear facility sites for seismic hazard assessments	2020	ANS	47
12	2.29	Probabilistic Seismic Hazards Analysis	2020	ANS	67
13	6.1.1	Photon and Neutron Fluence-to-Dose Conversion Coefficients	2020	ANS	19
14	51.10	Auxiliary feedwater system for pressurized water reactors	2020	ANS	41
15	54.1	Nuclear Safety Criteria and Design Process for Sodium Fast Reactor Nuclear Power Plants	2020	ANS	48
16	57.8	Fuel Assembly Identification	2020	ANS	36
17	58.8	Time Response Design Criteria for Safety-Related Operator Actions	2019	ANS	23
18	2.8	Probabilistic Evaluation of External Flood Hazards for Nuclear Facilities	2019	ANS	81
19	8.23	Nuclear Criticality Accident Emergency Planning and Response	2019	ANS	40
20	16.1	Measurement of the leachability of solidified low-level radioactive wastes by a short-term test procedure	2019	ANS	37
21	19.1	nuclear data sets for reactor design calculations	2019	ANS	25
22	19.6.1	Reload Startup Physics Tests for Pressurized Water Reactors	2019	ANS	36
23	3.5	Nuclear Power Plant Simulators for Use in Operator Training and Examination	2018	ANS	35
24	57.3	Design Requirements for New Fuel Storage Facilities at Light Water Reactor Plants	2018	ANS	20
25	2.6	Guidelines for Estimating Present & amp; Projecting Future Population Distributions Surrounding Nuclear Facility Sites	2018	ANS	40
26	2.10	Criteria for Retrieval, Processing, Handling, and Storage of Records from Nuclear Facility Seismic Instrumentation	2017	ANS	28
27	2.15	criteria for modeling and calculating atmospheric dispersion of routine radiological releases from nuclear facilities	2017	ANS	64
28	2.26	categorization of nuclear facility structures, systems, and components for seismic design	2017	ANS	29
29	3.2	American National Standard managerial, administrative, and quality assurance controls for the operational phase of nuclear power plants	2017	ANS	36
30	8.3	Criticality accident alarm system	2017	ANS	26
31	8.5	Use of borosilicate-glass Raschig rings as a neutron absorber in solutions of fissile material	2017	ANS	20
32	8.6	Safety in conducting subcritical neutron-multiplication measurements in situ	2017	ANS	12
33	8.7	Nuclear Criticality Safety in the Storage of Fissile Materials	2017	ANS	28
34	16.1	Measurement of the leachability of solidified low-level radioactive wastes by a short-term test procedure	2017	ANS	40

35	19.3.4	The Determination of Thermal Energy Deposition Rates in Nuclear Reactors	2017	ANS	15
36	19.3	Steady-state neutronics methods for power reactor analysis	2017	ANS	38
37	8.24	validation of neutron transport methods for nuclear criticality safety calculations	2017	ANS	34
38	19.4	Validation of Neutron Tranport Methods for Nuclear Criticality Safety Calculations	2017	ANS	34
39	19.11	Calculation and Measurement of the Moderator Temperature Coefficient of Reactivity for Water Moderated Power Reactors	2017	ANS	36
40	57.8	Fuel Assembly Identification	2017	ANS	16
41	19.11	Calculation and Measurement of the Moderator Temperature Coefficient of Reactivity for Pressurized Water Reactors	2017	ANS	36
42	55.1	Solid Radioactive Waste Processing System for Light-Water-Cooled Reactor Plants	2017	ANS	52
43	58.8	Time Response Design Criteria for Safety-Related Operator Actions	2017	ANS	22
44	58.14	Safety and Pressure Integrity Classification Criteria for Light Water Reactors	2017	ANS	77
45	56.8	Containment System Leakage Testing Requirements	2016	ANS	42
46	40.37	Mobile low-level radioactive waste processing systems	2016	ANS	49
47	57.10	Design Criteria for Consolidation of LWR Spent Fuel	2016	ANS	46
48	53.1	nuclear safety design process for modular helium-cooled reactor plants	2016	ANS	135
49	2.2	Earthquake Instrumentation Criteria for Nuclear Power Plants	2016	ANS	44
50	2.3	Estimating Tornado, Hurricane, and Extreme Straight Line Wind Characteristics at Nuclear Facility Sites	2016	ANS	19
51	15.4	Selection and training of personnel for research reactors	2016	ANS	24
52	15.11	radiation protection at research reactor facilities	2016	ANS	42
53	18.1	Radioactive Source Term for Normal Operation of Light Water Reactors	2016	ANS	26
54	2.23	Nuclear Plant Response to an Earthquake	2016	ANS	59
55	19.6.1	Reload Startup Physics Tests for Pressurized Water Reactors	2016	ANS	33
56	19.10	Methods for Determining Neutron Fluence in BWR and PWR Pressure Vessel and Reactor Internals	2016	ANS	14
57	15.2	Quality Control for Plate-Type Uranium-Aluminum Fuel Elements	2016	ANS	20
58	8.22	Nuclear Criticality Safety Based on Limiting and Controlling Moderators	2016	ANS	15
59	8.26	Criticality safety engineer training and qualification program	2016	ANS	11
60	10.4	verification and validation of non-safety-related scientific and engineering computer programs for the nuclear industry	2016	ANS	38
61	10.5	Accommodating user needs in scientific and engineering computer software development	2016	ANS	14
62	8.12	nuclear criticality control and safety of plutonium-uranium fuel mixtures outside reactors	2016	ANS	22
63	8.14	use of soluble neutron absorbers in nuclear facilities outside reactors	2016	ANS	11
64	6.4.2	Specification for Radiation Shielding Materials	2016	ANS	20
65	6.4	Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power Plants	2016	ANS	96
66	2.27	criteria for investigations of nuclear facility sites for seismic hazard assessments	2016	ANS	33
67	2.29	Probabilistic Seismic Hazards Analysis	2016	ANS	42
68	2.17	evaluation of subsurface radionuclide transport at commercial nuclear power plants	2016	ANS	42
69	2.21	Criteria for Assessing Atmospheric Effects on the Ultimate Heat Sink	2016	ANS	15
70	8.20	Nuclear criticality safety training	2015	ANS	16
71	9001	Quality management systems ù Requirements	2015	ANS	41
72	15.16	Emergency planning for research reactors	2015	ANS	21
73	10.8	Non-Real-Time, High-Integrity Software for the Nuclear Industry User Requirements	2015	ANS	31
74	8.27	Burnup Credit for LWR Fuel	2015	ANS	25
75	8.10	Criteria for nuclear criticality safety controls in operations with shielding and confinement	2015	ANS	15

76	6.6.1	Calculation and measurement of direct and scattered gamma radiation from LWR nuclear power plants	2015	ANS	47
77	3.11	Determining Meteorological Information at Nuclear Facilities	2015	ANS	57
78	2.30	Criteria for Assessing Tectonic Surface Fault Rupture and Deformation at Nuclear Facilities	2015	ANS	59
79	58.9	Single failure criteria for light water reactor safety-related fluid systems	2015	ANS	16
80	59.51	Fuel Oil Systems for Safety-Related Emergency Diesel Generators	2015	ANS	20
81	59.52	Lubricating Oil Systems for Safety-Related Emergency Diesel Generators	2015	ANS	18
82	RA-S-1.2 TRIAL USE	(Trial Use) Probabilistic Risk Assessment Standard for Advanced Non-LWR Nuclear Power Plants	2014	ANS	96
83	58.22 TRIAL USE	(Trial Use) Requirements for Low Power and Shutdown Probabilistic Risk Assessment	2014	ANS	299
84	3.1	Selection, Qualification, and Training of Personnel for Nuclear Power Plants	2014	ANS	36
85	5.1	Decay Heat Power in Light Water Reactors	2014	ANS	61
86	8.1	Nuclear Criticality Safety in Operations with Fissionable Materials Outside Reactors	2014	ANS	26
87	8.15	Nuclear criticality control of special actinide elements	2014	ANS	28
88	8.19	Administrative Practices for Nuclear Criticality Safety	2014	ANS	14
89	58.16	Safety Categorization and Design Criteria for Nonreactor Nuclear Facilities	2014	ANS	51
90	58.22	Requirements for Low Power and Shutdown Probabilistic Risk Assessment	2014	ANS	299
91	RA-S-1.2	Severe Accident Progression and Radiological Release (Level 2) PRA Standard for Nuclear Power Plant Applications for Light Water Reactors (LWRs)	2014	ANS	96
92	8.17	Criticality Safety Criteria for the Handling, Storage, and Transportation of LWR Fuel Outside Reactors	2014	ANS	11
93	14.1	Operation of Fast Pulse Reactors	2014	ANS	13
94	15.1	The development of technical specifications for research reactors	2013	ANS	24
95	15.8	Quality assurance program requirements for research reactors	2013	ANS	20
96	5.10	Airborne Release Fractions at Non-Reactor Nuclear Facilities - Includes Appendix A: 2013, Appendix B and Appendix C	2013	ANS	39
97	2.15	criteria for modeling and calculating atmospheric dispersion of routine radiological releases from nuclear facilities	2013	ANS	64
98	3.4	medical certification and monitoring of personnel requiring operator licenses for nuclear power plants	2013	ANS	44
99	6.1.2	Neutron and Gamma-Ray Cross Sections for Nuclear Radiation Protection Calculations for Nuclear Power Plants	2013	ANS	18
100	10.7	non-real-time, high-integrity software for the nuclear industry-developer requirements	2013	ANS	30
101	RA-S-1.4 TRIAL USE	(Trail Use) Severe Accident Progression and Radiological Release (Level 2) PRA Standard for Nuclear Power Plant Applications for Light Water Reactors (LWRs)	2013	ANS	501
102	15.21	format and content for safety analysis reports for research reactors	2012	ANS	35
103	41.5	verification and validation of radiological data for use in waste management and environmental remediation	2012	ANS	57
104	3.2	American National Standard managerial, administrative, and quality assurance controls for the operational phase of nuclear power plants	2012	ANS	36
105	2.21	Criteria for Assessing Atmospheric Effects on the Ultimate Heat Sink	2012	ANS	15
106	8.23	Nuclear Criticality Accident Emergency Planning and Response	2012	ANS	31
107	19.1	nuclear data sets for reactor design calculations	2011	ANS	17
108	8.21	use of fixed neutron absorbers in nuclear facilities outside reactors	2011	ANS	13
109	58.14	Safety and Pressure Integrity Classification Criteria for Light Water Reactors	2011	ANS	77
110	2.3	Estimating Tornado, Hurricane, and Extreme Straight Line Wind Characteristics at Nuclear Facility Sites	2011	ANS	19
111	5.4	Method for calculating the fractional release of volatile fission products from oxide fuel	2011	ANS	20
112	53.1	nuclear safety design process for modular helium-cooled reactor plants	2011	ANS	135

113	19.3	Steady-state neutronics methods for power reactor analysis	2011	ANS	38
114	19.6.1	Reload Startup Physics Tests for Pressurized Water Reactors	2011	ANS	33
115	2.17	evaluation of subsurface radionuclide transport at commercial nuclear power plants	2010	ANS	42
116	57.5 ERTA	Light Water Reactors Fuel Assembly Mechanical Design and Evaluation	2010	ANS	0
117	10.2	Portability of Scientific and Engineering Software	2009	ANS	15
118	3.5	Nuclear Power Plant Simulators for Use in Operator Training and Examination	2009	ANS	33
119	19.10	Methods for Determining Neutron Fluence in BWR and PWR Pressure Vessel and Reactor Internals	2009	ANS	14
120	40.37	Mobile low-level radioactive waste processing systems	2009	ANS	49
121	15.11	radiation protection at research reactor facilities	2009	ANS	34
122	15.16	Emergency planning for research reactors	2008	ANS	16
123	8.27	Burnup Credit for LWR Fuel	2008	ANS	13
124	10.4	verification and validation of non-safety-related scientific and engineering computer programs for the nuclear industry	2008	ANS	38
125	2.27	criteria for investigations of nuclear facility sites for seismic hazard assessments	2008	ANS	33
126	2.29	Probabilistic Seismic Hazards Analysis	2008	ANS	42
127	58.3	Physical Protection for Nuclear Safety-Related Systems and Components	2008	ANS	47
128	51.10	Auxiliary feedwater system for pressurized water reactors	2008	ANS	33
129	8.23	Nuclear Criticality Accident Emergency Planning and Response	2007	ANS	31
130	8.24	validation of neutron transport methods for nuclear criticality safety calculations	2007	ANS	28
131	8.26	Criticality safety engineer training and qualification program	2007	ANS	11
132	15.4	Selection and training of personnel for research reactors	2007	ANS	18
133	15.1	The development of technical specifications for research reactors	2007	ANS	24
134	6.3.1	Program for Testing Radiation Shields in Light Water Reactors (LWR)	2007	ANS	21
135	10.5	Accommodating user needs in scientific and engineering computer software development	2006	ANS	14
136	6.4.2	Specification for Radiation Shielding Materials	2006	ANS	20
137	6.4	Nuclear Analysis and Design of Concrete Radiation Shielding for Nuclear Power Plants	2006	ANS	96
138	8.19	Administrative Practices for Nuclear Criticality Safety	2005	ANS	10
139	3.11	Determining Meteorological Information at Nuclear Facilities	2005	ANS	44
140	5.1 ERTA	Decay Heat Power in Light Water Reactors	2005	ANS	1
141	5.1	Decay Heat Power in Light Water Reactors	2005	ANS	51
142	57.1	Design Requirements for Light Water Reactor Fuel Handling Systems	2005	ANS	22
143	2.26	categorization of nuclear facility structures, systems, and components for seismic design	2004	ANS	29
144	8.17	Criticality Safety Criteria for the Handling, Storage, and Transportation of LWR Fuel Outside Reactors	2004	ANS	13
145	8.14	use of soluble neutron absorbers in nuclear facilities outside reactors	2004	ANS	11
146	14.1	Operation of Fast Pulse Reactors	2004	ANS	15
147	16.1	Measurement of the leachability of solidified low-level radioactive wastes by a short-term test procedure	2003	ANS	42
148	19.1	nuclear data sets for reactor design calculations	2002	ANS	17
149	19.3.4	The Determination of Thermal Energy Deposition Rates in Nuclear Reactors	2002	ANS	18
150	56.8	Containment System Leakage Testing Requirements	2002	ANS	41
151	2.23	Nuclear Plant Response to an Earthquake	2002	ANS	35
152	58.9	Single failure criteria for light water reactor safety-related fluid systems	2002	ANS	16
153	1	Conduct of Critical Experiments	2000	ANS	10

154	10.2	Portability of Scientific and Engineering Software	2000	ANS	15
155	15.2	Quality Control for Plate-Type Uranium-Aluminum Fuel Elements	1999	ANS	18
156	5.10	Airborne Release Fractions at Non-Reactor Nuclear Facilities - Includes Appendix A: 2013, Appendix B and Appendix C	1998	ANS	39
157	8.7	Nuclear Criticality Safety in the Storage of Fissile Materials	1998	ANS	28
158	8.1	Nuclear Criticality Safety in Operations with Fissionable Materials Outside Reactors	1998	ANS	26
159	59.52	Lubricating Oil Systems for Safety-Related Emergency Diesel Generators	1998	ANS	18
160	59.51	Fuel Oil Systems for Safety-Related Emergency Diesel Generators	1997	ANS	20
161	8.3	Criticality accident alarm system	1997	ANS	30
162	8.22	Nuclear Criticality Safety Based on Limiting and Controlling Moderators	1997	ANS	20
163	19.11	Calculation and Measurement of the Moderator Temperature Coefficient of Reactivity for Water Moderated Power Reactors	1997	ANS	28
164	8.5	Use of borosilicate-glass Raschig rings as a neutron absorber in solutions of fissile material	1996	ANS	20
165	57.5	Light Water Reactors Fuel Assembly Mechanical Design and Evaluation	1996	ANS	26
166	57.10	Design Criteria for Consolidation of LWR Spent Fuel	1996	ANS	44
167	57.8	Fuel Assembly Identification	1995	ANS	16
168	58.11	Design criteria for safe shutdown following selected design basis events in light water reactors - Addendum:07/23/2002	1995	ANS	19
169	8.21	use of fixed neutron absorbers in nuclear facilities outside reactors	1995	ANS	13
170	15.8	Quality assurance program requirements for research reactors	1995	ANS	20
171	58.8	Time Response Design Criteria for Safety-Related Operator Actions	1994	ANS	23
172	55.4	Gaseous Radioactive Waste Processing Systems for Light Water Reactor Plants	1993	ANS	37
173	57.1	Design Requirements for Light Water Reactor Fuel Handling Systems	1992	ANS	22
174	55.1	Solid Radioactive Waste Processing System for Light-Water-Cooled Reactor Plants	1992	ANS	50
175	58.3	Physical Protection for Nuclear Safety-Related Systems and Components	1992	ANS	47
176	51.10	Auxiliary feedwater system for pressurized water reactors	1991	ANS	33
177	8.20	Nuclear criticality safety training	1991	ANS	16
178	8.12	nuclear criticality control and safety of plutonium-uranium fuel mixtures outside reactors	1987	ANS	28
179	6.6.1	Calculation and measurement of direct and scattered gamma radiation from LWR nuclear power plants	1987	ANS	44
180	6.3.1	Program for Testing Radiation Shields in Light Water Reactors (LWR)	1987	ANS	21
181	8.6	Safety in conducting subcritical neutron-multiplication measurements in situ	1983	ANS	12
182	8.10	Criteria for nuclear criticality safety controls in operations with shielding and confinement	1983	ANS	16
183	8.15	Nuclear criticality control of special actinide elements	1981	ANS	26

Hercules Ebooks Institute

www.herculesebooks.com info@herculesebooks.com +989141908737